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Diffusion and sorption behavior of HTO, Cs, I and U in mortar

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Due to the accident of Fukushima Daiichi Nuclear Power Plant (NPP), a large amount of contaminated water has been generated in such as the reactor building. Concrete materials of the building have been in contact with contaminated water for a long time, therefore radionuclides in contaminated water may be penetrating into these concrete materials. For developing the plans of decommissioning of NPP and waste management including decontamination and disposal, it is important to estimate radionuclides inventory and concentration distribution in the concrete materials. For this purpose, the present study focuses on understanding the sorption and diffusion behavior and related mechanisms in the ordinary Portland cement (OPC) mortar. Effective diffusion coefficients (D_e) and sorption distribution coefficients (D_e) in OPC mortar for HTO, D_e , I and D_e 0 values of tracers derived were measured by the through-diffusion and batch sorption experiments. As a result, D_e 0 values of tracers derived were in the sequence of HTO D_e 1 implying that cation exclusion effects may be important mechanisms in OPC mortar. D_e 1 values derived from diffusion and sorption experiments were consistently in the sequence of D_e 2 in however, the near-surface disturbance in diffusion samples and crushed effect in batch sorption samples must be considered for D_e 2 valuation for high-sorbing radionuclides such as D_e 3.

Keywords: Fukushima Daiichi nuclear power plant; contaminated concrete; uranium; cesium; diffusion; sorption

1. Introduction

Due to the accident of Fukushima Daiichi (1F) Nuclear Power Plant (NPP), a large amount of concrete wastes like rubble contaminated by radioactive cesium has been generated. Moreover, it is assumed that a large amount of concrete waste will be generated by the cleanup and decommissioning activities [1]. Especially, concrete materials of the reactor building have been in contact with contaminated water for a long time. Therefore, it is considered that radionuclides (RN) in contaminated water may be penetrating into these concretes. For consideration of decommissioning and disposal, it is important to estimate RNs inventory and concentration distribution in the concrete materials. However, a lot of diffusion data of water in concrete has been acquired using HTO [2,3], but the diffusion data of RNs such as Cs and U in concrete is very limited [4]. In this study, sorption distribution coefficients (K_d) and effective diffusion coefficients (De) of key RNs, HTO, Cs, I and U, in OPC (ordinary Portland cement) mortar were measured by the through-diffusion and batch sorption experiments. HTO was used for understanding the fundamental diffusion characteristics. Cs and I were

used as typical cation and anion, respectively. U(VI) was used as a representative of more complex actinides. Sorption and diffusion behavior and related mechanisms in OPC mortar were evaluated to predict the RNs inventory and distribution in the concrete materials.

2. Experimental

2.1. Mortar samples

OPC mortar samples used in this study were selected based on the following reasons. The OPC was selected as a standard cement and due to the existence of related data. Migration of RNs in concrete is affected by the properties of aggregate in addition to the properties of cement paste. Especially, the sorption properties for aggregates strongly influence the time until the RNs breakthrough by diffusion in concrete, and it was estimated that the breakthrough could not be obtained for high-sorbing tracers. Therefore, in this study, concrete was produced using limestone as aggregate having weak sorption for RNs. In addition, if the thickness of the sample was cut thicker than the coarse aggregate's size, the diffusion experiments would need a very long time to obtain the nuclide break through, so mortar samples using diffusion experiments were made

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without coarse aggregates. The mortar sample was prepared by mixing with a predetermined formulation including coarse aggregate and then removing the coarse aggregate and retaining the fine aggregate by a wet screening method. The mortar samples were cured for 4 weeks, the porosity and density was measured by mercury intrusion porosimetry, and the specific surface area was measured by BET adsorption method (**Table 1**).

Table 1. Composition and characteristic of the OPC mortar sample

| Amount (kg m ⁻³) | | | | | |
|------------------------------|--------|-------------------------|----------------|-----------------------------------|------------------|
| Water | Cement | | Fine aggregate | | Coarse aggregate |
| 175 | | 318 882 | | 2 | 943 |
| Measured characteristic | | | | | |
| Porosity | | Bulk density | | Surface area (BET) | |
| 14.2 % | | 2.32 Mg m ⁻³ | | $12.5 \text{ m}^2 \text{ g}^{-1}$ | |

2.2. Diffusion experiments

Diffusion experiments were conducted at room temperature and in an argon atmosphere in order to avoid the dissolution of carbon dioxide in solution and the precipitation of CaCO₃ at surface of OPC mortar. OPC mortars were cut into disk shapes (50mm in diameter and 5mm in thickness) were set to a through-diffusion type cell (Figure 1). Two reservoirs (inlet and outlet) of the diffusion cell were filled by test solution, and the mortar samples were pre-saturated by evacuation. The test solution used is deionized water equilibrated with mortar, and its pH was expected to be kept at 12.5 by dissolution of Ca(OH)₂ in the mortar [5]. The initial tracer concentrations of HTO, Cs, I and U(VI) were 13.9 kBq mL⁻¹, 8.1 x 10⁻⁸ mol mL⁻¹, 9.8 x 10⁻⁸ mol mL⁻¹ and 2.5 x 10⁻⁸ mol mL⁻¹. All diffusion experiments were conducted in duplicate under the same conditions. Tracer concentrations in both reservoirs were continuously monitored until the concentration increase in the outlet reservoir approached near linear. These tracer concentrations were measured by using Liquid Scintillation Counter (LSC: Perkin Elmer) for HTO and Inductively Coupled Plasma Mass Spectrometer (ICP-MS: Agillent) for Cs, I and U. At the end of the

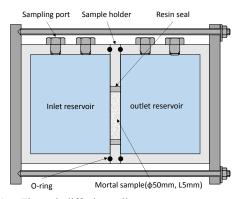


Figure 1. Through diffusion cell.

diffusion test, the diffusion cells for Cs and U(VI) were disassembled and the mortar samples were drilled, and the obtained fine powders were leached with HNO₃, and the concentrations of tracer in the leachate were measured by ICP-MS.

2.3. Batch sorption experiments

Batch sorption experiments for Cs, I and U(VI) were conducted at room temperature and in an argon atmosphere in the same way as the diffusion experiments. OPC mortars were crushed to powder (<0.25mm) and were immersed in polypropylene bottles with test solution equilibrated with mortar at a liquid to solid ratio of 10 mL g⁻¹ (3 g solid in 30 mL solution). The initial tracer concentrations of Cs, I and U(VI) were set equal to the diffusion experiments. The batch experiments were carried out for 28 days and the sorption was confirmed to have reached equilibrium. The solution containing these tracers were passed through a 0.45 μm membrane filter (PTFE; Advantec[□]) and the concentrations of tracer were measured by ICP-MS. All sorption experiments were conducted in triplicate, and a blank test without OPC mortar was also conducted.

3. Results and discussion

3.1. Diffusion experiments

The diffusion and sorption characteristics of OPC mortars were calculated from the results of the through diffusion experiments using Fick's second law. A rigorous analytical solution that considers a decrease concentration in inlet reservoir and an increase concentration in outlet reservoir can be described as follows [6];

$$\begin{split} C(x,t) &= \frac{C_1}{\delta + \gamma + 1} - \\ &2C_1 \sum_{m=0}^{\infty} \frac{\exp\left(-\frac{D_e \cdot \varphi_m^2}{\alpha \cdot L^2} \cdot t\right) \cdot \left[\delta \cdot \cos\left(\varphi_m \cdot \frac{L - x}{L}\right) - \gamma \cdot \varphi_m \cdot \sin\left(\varphi_m \cdot \frac{L - x}{L}\right)\right]}{\left[\gamma \cdot \varphi_m^2 - \delta(\delta + \gamma + 1)\right] \cdot \cos(\varphi_m) + \left(\delta \gamma + \delta + 2\gamma\right) \cdot \varphi_m \cdot \sin(\varphi_m)} \end{split} \tag{1}$$

where C_1 is the initial concentration of inlet reservoir (Bq mL⁻¹ or mol mL⁻¹), L is the thickness of a sample (m), α is the capacity factor (-), x is the distance (m), t is the time (s), $\delta = (\alpha \cdot A \cdot L)/V_{in}$, $\gamma = V_{out}/V_{in}$ and ϕ_m is the root of the following equation:

root of the following equation:

$$\tan(\varphi) = \frac{\delta \cdot (\gamma + 1) \cdot \varphi}{\gamma \cdot \varphi^2 - \delta^2}$$
(2)

where A is the cross-sectional area of a sample (m^2) and V_{in} and V_{out} are the volumes of the inlet and outlet reservoirs (m^3) .

Breakthrough curves in the outlet reservoir for HTO, Cs and I were obtained (**Figure 2**). D_e of HTO, D_e and K_d values of Cs and I were calculated by fitting to change for tracer's concentration both of inlet and outlet reservoirs using a rigorous analytical solution. Calculated D_e values of HTO were $4.7 \sim 5.1 \times 10^{-12} \text{ m}^2 \text{ s}^{-1}$ and D_e values of Cs were $0.69 \sim 1.1 \times 10^{-12} \text{ m}^2 \text{ s}^{-1}$ and

 D_e values of I were 3.5 \sim 6.4 x 10^{-12} m² s⁻¹. The D_e values of HTO are exactly consistent with an empirical relationship between porosity and D_e of OPC [2]. D_e values of I were almost identical as HTO. D_e value of Cs was slightly lower than D_e of HTO and I. It suggests that the diffusion of cation in the pore of mortar and the positive charge on the surface of cement hydrate [7] cause an electrostatic interaction. Although there is some scatter in the data during early stages, the concentration of I in the inlet reservoir indicated a decrease due to the sorption onto the cement hydrate.

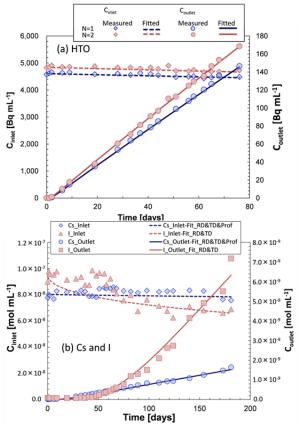


Figure 2. Concentration of HTO, Cs, and I tracers.

In the U(VI) case, the breakthrough curves could not be obtained, then depth profiles were analyzed. The depth profiles of Cs were also measured in the same way as U(VI). The measured depth profile is shown in **Figure 3**. Typical dual profiles were observed for U and Cs. One of them has a high concentration gradient near the surface on the inlet reservoir (near profile), the other one has a lower concentration gradient extending into the sample (far profile). These are similar to existing observations for Cs diffusion in mortar [8] and granitic rock [9]. These suggest that the diffusion behavior is different between the near-surface and the deeper parts.

The profile of U(VI) in the sample showed more than 90 % of U remained in the area from the surface to the depth of 1 mm. Concentration of U(VI) at near the surface decreased more than one order of magnitude. These might be strong sorption at the surface of the

mortar sample. Concentration of Cs near the surface decreased to about half of the level of the surface. This might be weak sorption compared with U(VI).

 D_e values of these RNs shown in **Figure 4** were in the sequence of HTO \approx I > Cs > U. D_e value of U(VI) was estimated as about 5 x 10^{-13} m² s¹ from the far profile. The D_e values of U(VI) was one order lower than D_e of HTO and I, and slightly lower than D_e of Cs. Regarding D_e values of Cs, the D_e calculated from the breakthrough curve and D_e obtained from the far profile were almost identical. Likewise for U(VI), the D_e value calculated from the breakthrough curve is expected to be close to the D_e value calculated from the far profile.

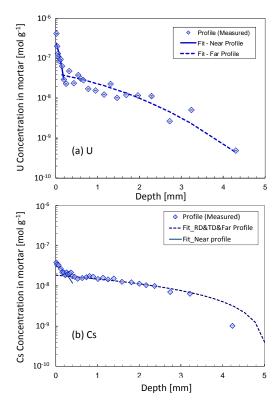


Figure 3. Depth profiles of Cs and U(VI) in OPC mortar.

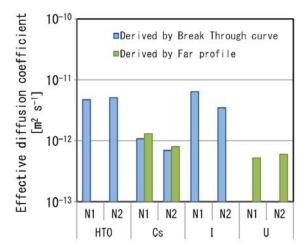


Figure 4. The D_e of HTO, Cs, I, and U(VI) in OPC mortar, obtained from through diffusion experiments.

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3.2. Batch sorption experiments

The K_d values were determined from the following equation

$$K_d = \frac{(c_{ini} - c_{eq})}{c_{eq}} \times \frac{v}{w} \tag{3}$$

where C_{ini} and C_{eq} are the equilibrium concentrations of tracers in the test batch and blank batch (mol m⁻³), respectively. V is the volume of liquid phase (m³), and W is the mass of solid phase (kg). The K_d values are shown in Figure 5, compared with the K_d values calculated from the results of diffusion experiments. The K_d values of Cs, I and U were in the sequence of U > I >Cs in both of the batch sorption experiments and fitting to diffusion experiments. This sequence is similar with a previous report [7]. The K_d values of U(VI) had a noticeable difference between batch sorption and diffusion experiments. K_d values derived from batch tests (Kd = 7.8 kg m^{-3}) were higher by more than two orders of magnitude than the diffusion-derived K_d values ($K_d = 0.011 \text{ kg m}^{-3}$). The K_d value of Cs is in the sequence of the far profile < near profile < batch sorption. This trend was also observed for U(VI), however, in the U(VI) case, the difference between batch sorption and the diffusion experiment is significantly large. From this, it is thought that U(VI) sorption is strongly affected by the disturbances at near-surface of diffusion samples and crushed samples.

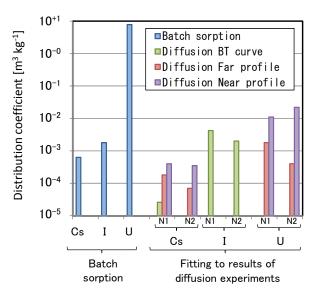


Figure 5. The distribution coefficients of Cs, I, and U(VI), obtained from batch sorption and diffusion experiments.

4. Conclusions

The D_e and K_d value in the OPC mortar for HTO, Cs, I and U were measured by the through-diffusion and batch sorption experiments. As a result, the influence of charge of diffusion species on diffusion characteristics was confirmed in cement mortar which is complex of plural minerals and hydrates. Trends of the D_e value were in the sequence of HTO \approx I > Cs, it might be

suggested that the positive charge on the surface of cement hydrate cation effected for diffusion of Cs. And the tendency of K_d values were consistent with the batch sorption and diffusion experiment then the batch experiments is also effective, but depending on the high-sorbing RNs such as U(VI), it is necessary attention to inconsistency of the batch sorption and diffusion due to near-surface disturbance in diffusion samples and crushed effect in batch sorption samples. The sorption and diffusion trends and their mechanisms observed here are important to evaluate RNs inventory and concentration distribution in the concrete materials, and should be reflected for developing the plans of decommissioning NPPs and waste disposal.

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