
ARTICLE

Measurement of neutron energy spectra behind shields for quasi-monoenergetic neutrons generated by 246-MeV and 389-MeV protons using a Bonner sphere spectrometer

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Neutron energy spectra behind concrete and iron shields were measured for quasi-monoenergetic neutrons above 200 MeV. Quasi-monoenergetic neutrons were produced by the ${}^7\text{Li}(p,xn)$ reaction with 246-MeV and 389-MeV protons. A Bonner sphere spectrometer was used to obtain the neutron energy spectra. Shielding materials are concrete blocks with thicknesses from 25 cm to 300 cm and iron blocks with thicknesses from 10 cm to 100 cm. The neutron energy spectra behind the concrete and iron shields were obtained by the unfolding method using the MAXED code. Ambient dose equivalents were obtained as a function of a shield thickness successfully. These experimental data will be useful to perform accurate shielding design, benchmark calculation codes and evaluate neutron cross section data.

Keywords: *quasi-monoenergetic neutrons; Bonner sphere spectrometer; shielding experiments; concrete; iron; unfolding; dose distribution*

1. Introduction

Recently, high-energy and intense beam accelerators, such as the Japan Proton Accelerator Research Complex (J-PARC) are used for various studies and industries. In these facilities, secondary neutrons with energies above 100 MeV are produced around accelerators and beam lines by nucleon-nucleus and nucleus-nucleus reactions. For accurate shielding design of high energy accelerators, experiment data are indispensable to construction of these facilities from the point of view of radiation protection. High-energy neutron penetration data for main shielding materials (concrete and iron) are very important. However, the shielding experimental data for high energy neutrons above 100 MeV are still insufficient both in quality and in quantity as compared with those below 100 MeV [1-5].

In this study, we measured the neutron energy spectra behind concrete and iron shields using a Bonner sphere spectrometer (BSS) for quasi-monoenergetic neutrons

produced by the ${}^7\text{Li}(p,xn)$ reaction with 246-MeV and 389-MeV protons at the Research Center for Nuclear Physics (RCNP) of the Osaka University. The experimental data are used to investigate accuracy of calculation codes and evaluate neutron cross section data.

2. Experiments

2.1. Neutron source

Figure 1 shows a schematic view of a typical experimental arrangement. Shielding experiments were performed at the time-of-flight (TOF) beam course of RCNP. Quasi-monoenergetic neutrons with peak energies of 243.5 MeV and 386.6 MeV were produced by the ${}^7\text{Li}(p,xn)$ reaction by bombarding a 1-cm thick Li target with 246-MeV and 389-MeV protons from the AVF cyclotron and the ring cyclotron of RCNP. A neutron beam emitted in the forward direction was

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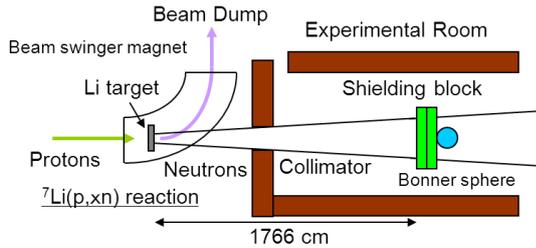


Figure 1. Schematic view of a typical experimental arrangement at RCNP.

extracted into the TOF room through an iron collimator of 12-cm wide and 10-cm high aperture embedded inside a concrete wall with a thickness of 15 cm. The proton beam penetrated through the target was guided to a beam dump using a beam swinger magnet. Neutron source spectra in the TOF room were measured with liquid scintillators by means of the TOF method [6].

2.2. Setup

Shielding materials are concrete blocks (2.33 g/cm³) with thicknesses from 25 cm to 300 cm and iron blocks (7.87 g/cm³) with thicknesses from 10 cm to 100 cm. The components of concrete are given in **Table 1**. The distance between the Li target and upstream surface of the shielding materials was 17.7 m. The BSS was contact with the backward surface of shielding material.

2.3. Bonner sphere spectrometer

Neutron energy spectra behind the shields were measured with the BSS [7]. The BSS consists of a ³He spherical proportional counter (CENTRONIC LTD, SP9, gas pressure: 21.3 kPa) and polyethylene (PE, 0.95 g/cm³) moderators with diameters from 7.62 cm to 24.1 cm. In addition, measurements were performed using inserting metal shells made of lead (457p: 10.2-cm-diameter PE sphere + 1.27-cm thick Pb shell + 2.54-cm thick PE shell) and copper (457c: 10.2-cm-diameter PE sphere + 1.27-cm-thick Cu shell + 2.54-cm thick PE shell) [8]. The response functions of BSS were simulated with the MCNPX code [9]. The JENDL-HE [10] file was used in the simulation. The responses at neutron energies of 144 keV, 565 keV, 5.0 MeV and 14.8 MeV were calibrated in the mono-energetic neutron standard fields [11] of the National Institute of Advanced Industrial Science and Technology. The calibrated and the calculated results are in good agreement within measurement uncertainties. Responses for 243.5-MeV and 386.6-MeV mono-energetic neutrons were also experimentally evaluated by the two-angle differential measurements, which was described in detail elsewhere [7]. **Figure 2** shows the response function of the BSS.

However, because the BSS was contacted with the surface of shielding block, actual response functions were affected by the neutron multi-scattering effect between the moderator of BSS and the shielding block, called ping-pong effect. The response function was also

Table 1. Components of concrete

Nuclide	Atomic density (10 ²² cm ⁻³)	Nuclide	Atomic density (10 ²² cm ⁻³)
H	1.47	Si	0.419
C	0.836	S	0.018
O	4.13	K	0.00375
Na	0.0194	Ca	0.987
Mg	0.0297	Fe	0.0192
Al	0.0725		

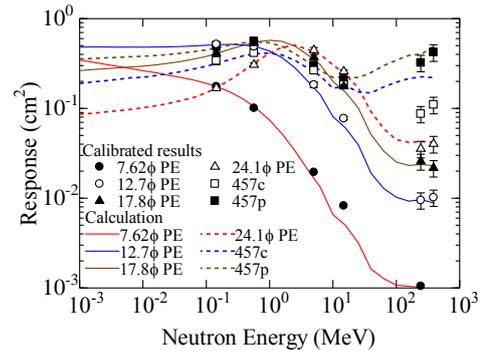


Figure 2. Calculated response function of the BSS. The calibrated results are also shown.

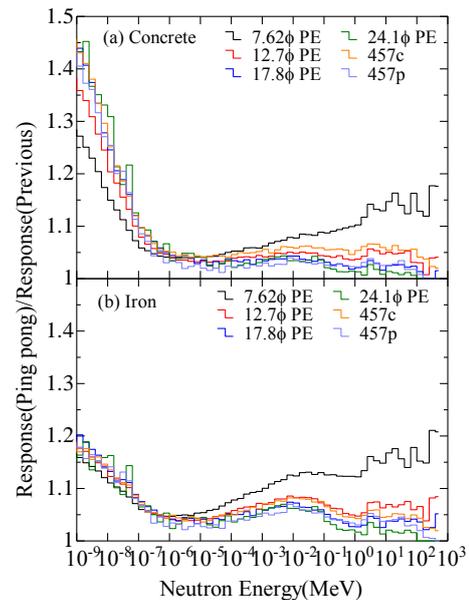


Figure 3. Ratio of the previous response function and the response function including the ping-pong effect for (a) concrete and (b) iron.

evaluated with the MCNPX code. **Figure 3** shows the ratio of the previous response function and the response function including the ping-pong effect. Figure 3 indicates the ping-pong effect for the concrete block is more conspicuous than that for the iron block in the low energy region. The response function including the ping-pong effect was used in the data analysis to obtain more precise results.

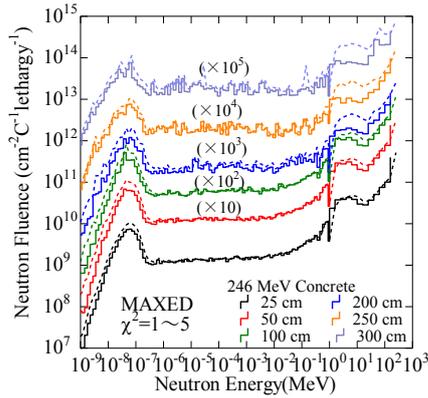


Figure 4. Measured neutron spectra behind the concrete shield using the 243.5-MeV quasi-monoenergetic neutrons. Dotted lines show the simulated spectra.

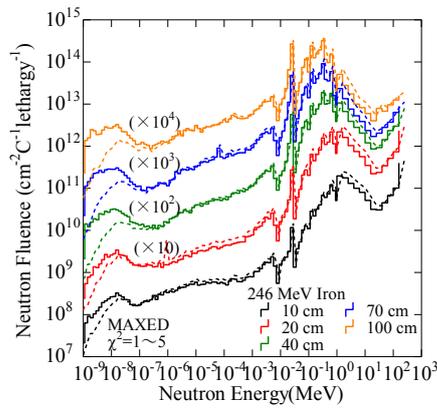


Figure 5. Measured neutron spectra behind the iron shield using the 243.5-MeV quasi-monoenergetic neutrons. Dotted lines show the simulated spectra.

3. Data analysis

Neutron energy spectra behind the shields were obtained by unfolding method using the MAXED code [12]. The measured neutron source energy spectra using the TOF method were used to simulate the neutron energy spectra behind the shielding materials in the MCNPX code with rather simple geometrical modeling without a floor and walls of the experimental room. The simulated neutron energy spectra were used for initial spectra in the MAXED code. From the neutron energy spectra with the BSS, the ambient dose equivalent was obtained using conversion coefficients based on the International Commission on Radiological Protection recommendations in 1990 [13, 14].

4. Results and discussion

Figures 4, 5, 6 and 7 show the neutron spectra penetrated behind the concrete or iron shields for the quasi-monoenergetic neutrons generated by the 246-MeV and 389-MeV protons. The neutron fluences are converted to the lethargy fluences at 1 m away from the Li target per proton beam charge (coulombs) in

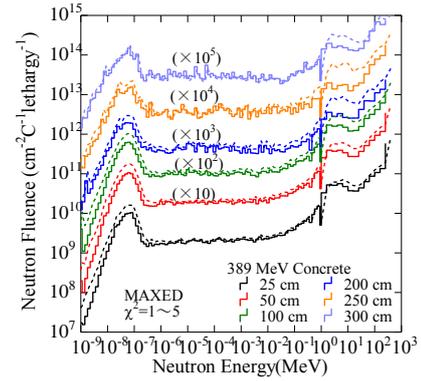


Figure 6. Measured neutron spectra behind the concrete shield using the 386.6-MeV quasi-monoenergetic neutrons. Dotted lines show the simulated spectra.

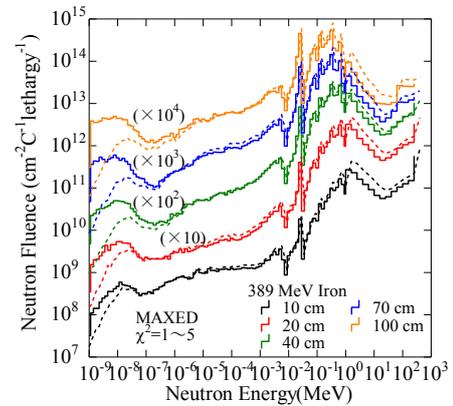


Figure 7. Measured neutron spectra behind the iron shield using the 386.6-MeV quasi-monoenergetic neutrons. Dotted lines show the simulated spectra.

figures 4, 5, 6 and 7. For the unfolding calculations, χ^2 static of the solution [12] is obtained from the measured counts and the predicted counts based on the initial spectrum and the response functions for each Bonner sphere. For all spectra, the χ^2 static shows values from 1 to 5, which indicates validity of the unfolded results. These figures indicate that thermal neutrons produced by slowing down in the shielding material are clearly observed. The thermal neutron component also includes room scattered neutrons. It is necessary to correct the contamination of scattered neutrons by calculations, which will be performed in future. Figures 8 and 9 show the preliminary results of neutron ambient dose equivalents per proton beam charge (coulombs) behind the concrete and iron shields as a function of the shield thickness, respectively. The neutron ambient dose equivalents are converted to those at 1 m away from the Li target in figures 8 and 9. In the shielding experiments, other measurements with a 25.4-cm-diameter and 25.4-cm-thick NE213 scintillator were also performed. The final results of neutron ambient dose equivalents and the attenuation length will be obtained after comparing with the experimental data obtained by the NE213 scintillator in fast energy region and calculated results in all energy region in future work.

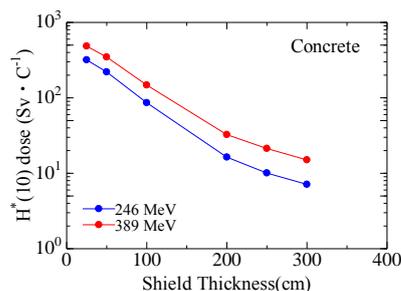


Figure 8. Ambient dose equivalents as a function of the shield thickness behind the concrete shield

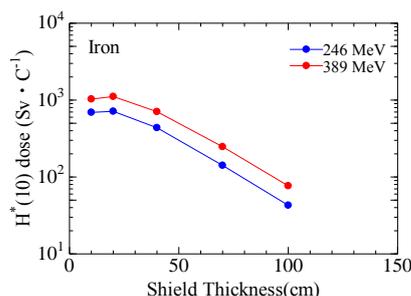


Figure 9. Ambient dose equivalents as a function of the shield thickness behind the iron shield

5. Conclusions

The neutron energy spectra were measured through the concrete and iron shields for the quasi-monoenergetic neutrons produced by the ${}^7\text{Li}(p,xn)$ reaction with the 246-MeV and 389-MeV protons. The neutron energy spectra behind the shielding blocks of concrete and iron were obtained successfully using the BSS, respectively. These experimental data will be useful to perform accurate shielding design and to investigate the accuracy of calculation codes and evaluate neutron cross section data.

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