Atomic Energy Society of Japan
Journal Standard Keyword List

14 MeV neutron
accelerator
accelerator-driven system
accident
accident management
accountability
accuracy
actinides
activation energy
activation method
ADS
adsorption
alpha particle
americium
analysis
angular distribution
annular flow
APR1400
benchmark
beta ray
blanket
BNCT
boiling transition
boiling water reactor
boron
boron carbide
breakthrough curve
burnup
burnup calculation
BWR
BWR type reactor
calculation
calibration
CANDU type reactor
canister
carbon
carbon dioxide
cesium
cesium 137
CFD
chromatography
cladding
CMPO
coated fuel particle
cobalt
cobalt 60
colloid
comparative evaluation
computational fluid dynamics
computer code
concentration dependence
consensus building
contaminated material
control rod
control rod material
corrosion
coupled-channel optical model
covariance
cracking
credibility
critical experiment

critical heat flux
critical power
criticality
criticality accident
criticality calculation
cross section
curium
cyclic voltammetry
database
decision Making
decommissioning
decommissioning waste
decontamination
density
density wave oscillation
deposition
design
deuterium
diffusion
diffusion coefficient
dismantling
dismantling activities
dispersion
dissolution
distribution coefficient
documentation
dose rate
dosimetry
double differential cross section
dryout
economics
education
effective delayed neutron fraction
EGS
electrochemical corrosion potential
electrochemistry
electron
electrorefining
ENDF/B-VI
energy distribution
entrainment
environment
error
ethics
europium
evaluation
experiment
experimental data
extraction
fabrication
fast breeder reactor
fast reactor
FBR
FBR type reactors
feasibility study
film boiling
finite difference method
finite element method
first wall
fission gas release
fission product
flow rate
fluid flow

fracture
fragmentation
fuel
fuel assembly
fuel cladding
fuel cycle
fuel debris
fuel pellet
fuel rod
Fukushima Daiichi Nuclear Power Plant
fusion
fusion reactor
gadolinium
gamma radiation
gamma ray
geological disposal
glass waste form
gold 197
grain boundary
heat flux
heat transfer
high burnup
high burnup fuel
high level liquid waste
high level radioactive waste
high temperature
HTGR
HTGR type reactor
HTTR
hydrazine
hydride
hydrogen
hydrogen isotope
hydrogen peroxide
hydrogen production
hydrogen water chemistry
inert matrix
INES
information
inspection
intergranular stress corrosion cracking
international Relations
in-vessel retention
iodine
ion exchange
iron
irradiation
isotope effect
isotope ratio
isotope separation
ITER
JENDL
JENDL-3.2
JENDL-3.3
JENDL-4.0
JT60
justice
KUCA
lanthanide
laser
laser isotope separation
lead
lead-bismuth eutectic

light water reactor
liquid metal
lithium
lithium isotope
LMFBR type reactor
LOCA
loss of coolant
LWR type reactor
mass transfer
MCNP
measurement
mechanical property
melting
metallic fuel
method of characteristics
microstructure
migration
minor actinide
mixed oxide fuel
mixture
molten salt
Monte Carlo
Monte Carlo calculation
Monte Carlo method
MOX
MOX fuel
MPS method
MVP
natural circulation
natural convection
neptunium
neptunium 237
neutron
neutron capture
neutron capture cross section
neutron diffusion equation
neutron irradiation
neutron nuclear data
neutron source
neutron spectrum
nickel
nitric acid
Non-proliferation
NSRR
nuclear criticality safety
nuclear data
nuclear fuels
nuclear material
nuclear power plant
nuclear reactor
nuclear security
nuclear transmutation
nuclear waste treatment
nucleate boiling
numerical analysis
numerical simulation
numerical solution
ODS
operational transients
optimization
overpack
oxalic acid
oxidation

oxide film
oxygen
palladium
particle size
partitioning
passive safety
PCMI
performance
pH value
philosophy
PHITS
plutonium
plutonium oxide
policy
post irradiation examination
POST-BT
POST-DNB
power distribution
PRA
precipitation
pressure dependence
pressure drop
pressure wave
proliferation resistance
proton
PSA
public
pulse-height weighting technique
PWR type reator
quench
radiation chemistry
radiation damage
radiation dose
radiation effect
radiation protection
radiation shielding
radioactive inventory
radioactive waste
radioactive waste management
radioactivity
radiochemical analysis
radioisotope
radionuclide
radon
rare earth element
reaction rate
reactivity
reactivity initiated accident
reactor core
reactor kinetics
reactor physics
reactor safety
recovery
reliability
remote control
remote handling
reprocessing
resonance
resonance integral
response function
RIA
risk
risk Analysis

risk assessment
risk communication
risk management
robotics
rod bundle
safeguard
safety
safety analysis
safety assessment
seismic analysis
sensitivity
sensitivity analysis
sensitivity coefficient
separation
separation factor
separation process
severe accident
shielding
simulation
single phase flow
social
social responsibility
social technology
sociology
sodium
sodium cooled fast reactor
sodium-water reaction
solidification
solubility
solvent extraction
sorption
source term
spent fuel
spent fuel transportation cask
SRAC
stability
stainless steel
stakeholder
statistical model
steam generator
stress corrosion cracking
strontium
subchannel analysis
subcooling
subcriticality
supercritical pressure
supercritical water
surface tension
swelling
target
TBP
temperature dependence
temperature distribution
thermal conductivity
thermal diffusion column
thermal diffusion factor
thermal diffusivity
thermal hydraulics
thermal neutron
thermodynamics
thorium
Time-of-Flight method
TRACY

training
transmutation
transparency
transport
transport of radioactive waste
tritiated water
tritium
trust
two-fluid model
two-phase flow
uncertainty
unfolding
uptake
uranium
uranium 235
uranium 238
uranium dioxide
uranium oxide
uranium-bearing waste
validation
vapor explosion
verification
VHTR
void fraction
water
water chemistry
Xenon
X-ray
Zircaloy
Zircaloy-2
Zircaloy-4
zirconium
zirconium alloy
zirconium oxide